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A highly efficient neutron time-of-flight detector for inertial confinement fusion experiments
Calibration of time of flight detectors using laser-driven neutron source

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Calibration of three scintillators (EJ232Q, BC422Q, and EJ410) in a time-of-flight arrangement using a laser drive-neutron source is presented. The three plastic scintillator detectors were calibrated with gamma insensitive bubble detector spectrometers, which were absolutely calibrated over a wide range of neutron energies ranging from sub-MeV to 20 MeV. A typical set of data obtained simultaneously by the detectors is shown, measuring the neutron spectrum emitted from a petawatt laser irradiated thin foil. © 2015 AIP Publishing LLC. [http://dx.doi.org/10.1063/1.4923088]

I. INTRODUCTION

In recent years, there has been a growing interest in laser-driven sources for their high brightness, directionality, and compactness, which have shed a new light on the production of ultra-short (sub-ns) neutron pulses that can be used in many potential applications in science, 1 industry, 2 security, 3 and healthcare. 4,5 Since laser-driven neutron sources are primarily based on beam-fusion or spallation reactions initiated by fast ions, 6 the measurement of absolute neutron spectrum is crucial not only for the development and optimisation of the neutron sources but also for the study of the parent ions involved in the neutron generation process.

Among commonly used neutron diagnostics, those based on time-of-flight (TOF) arrangement are probably the most widely used for characterising the neutron spectra. Depending on the distance between the source and the detector, a TOF spectrometer can in principle resolve neutron energies ranging from sub-MeV to hundreds of MeV. However, the most unique feature of TOF scintillator detector lies in the capability to provide an absolute spectral measurement of fast neutrons, which are otherwise first moderated to lower energies in other types of neutron spectrometers resulting in a poor spectral information. 7 Most common TOF detectors are made of scintillators coupled to photo-multiplier tubes (PMTs), where the specifications of the scintillators and PMTs depend on the type and range of measurements sought. Plastic scintillators are the most common type of detectors employed for neutron detection due to their high dynamic range and ease of handling.

In order to obtain the complete spectral information from a TOF diagnostic, one would require an absolute calibration of the detector over a range of neutron energies. Traditionally, the scintillators are calibrated with continuous sources, such as 252Cf emitting neutrons via spontaneous fission reactions 8,9 by using pulse-height discrimination technique in multi-channel analyser. 10 However, the radioactive isotopes do not provide an accurate measurement of detector response due to their broad energy spectrum (for example, 0.1-10 MeV for 252Cf).

Since the scintillators fluoresce by the knock-on protons produced by neutron elastic scattering, their response to different energy neutrons exhibits a highly non-linear behaviour. In this paper, we report on an alternative way of calibrating the scintillator detectors by using an ultra-short burst of neutrons, which enables an energy resolved calibration by deploying the detectors in a TOF arrangement. High power laser driven pulsed neutron sources emit high flux of neutrons within a short burst duration (sub-ns). Therefore, using a detector at a suitably large distance (several meters) from the source temporally disperses the arrival of different energy neutrons on the detector. This provides an ideal situation for energy resolved calibration of the detector using a broadband source. This is unlike the pulse-height discrimination technique, where the neutron energy is identified from the height of the pulses generated by neutron induced scintillations, while assuming that each pulse is produced by a single hit of neutron. The laser driven source used in our case delivered high flux of neutrons of the order of 109 n/sr in a single shot, which was able to produce a strong signal in the TOF.

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detector for cross-calibration with the absolutely calibrated bubble detector spectrometers (BDSs) fielded in the same shot. The calibration technique presented here combines a series of experimental measurements and Monte Carlo simulations, which was employed to calibrate three detectors made of different plastic scintillators. As a typical detector consists of many parts, such as scintillator, light guide, and PMT, an absolute calibration requires specific knowledge of the detector design and characteristics of its different parts, such as the detector housing, light guide, and PMT. However, the non-linear response of the detector to different energy neutrons comes from the plastic scintillator itself, which originates from the difference in recoil proton spectra produced by different energy neutrons. The relative response of the scintillator over a range of neutron energy was obtained by Monte Carlo simulations, while benchmarking the results with the literature. The neutron spectra from the TOF data were obtained by using the energy dependent scintillator response, while taking into account the transmission of different energy neutrons through various objects along the line of sight to the detector. Finally, the detectors were calibrated by comparing the TOF neutron spectra with those obtained by the absolutely calibrated BDSs employed in the same shot.

The subject matter in this paper is organised in four sections. Section II illustrates a brief description of the experimental setup. Section III describes the basic principles of neutron detection by scintillation processes. Section IV outlines the numerical work carried out by employing Monte Carlo simulations in order to obtain the relative response of our scintillator to different energies of fast neutrons. Section V reports the final calibration obtained for the three scintillator detectors and typical neutron spectra obtained by different detectors.

II. EXPERIMENTAL SETUP

Calibration of the scintillator detectors was carried out using a laser driven neutron beam generated at VULCAN Petawatt target area of Rutherford Appleton Laboratory, STFC, UK. The 750fs FWHM laser pulse of Vulcan, with energy of 600 J, was focused on the target by an f/3 off-axis parabola delivering intensity in the range of 1 – 5 × 10^19 W cm^{-2}. Energetic ions were produced by irradiating the laser on deuterated plastic (C_2D_4) foils of various thicknesses, which were employed to generate neutrons through beam fusion reaction by interacting with either the bulk target ions or secondary C_2D_4 catcher target placed at a small distance (~cm) from the laser irradiated target.

A suite of diagnostics, such as scintillator TOF, BDS, activation samples, and CR39 stacks were employed around the target in order to characterise the neutron beam parameters. A schematic of a typical experimental setup used for cross-calibrating the scintillator detectors with the bubble detectors is shown in Fig. 1. The bubble detectors, manufactured and calibrated by BTIs (Bubble technology industries), were placed inside the interaction chamber at various distances (30-50 cm) from the neutron source in order to ensure a sufficient number of bubble formation. The scintillator detectors were placed outside the interaction chamber at a significantly larger distance from the neutron source to achieve high energy resolution by the TOF technique. The scintillator detectors were shielded appropriately by lead and plastic blocks for the purpose of reducing noise level mainly originated by gamma rays and thermal neutrons hitting the scintillators. Two 5 cm thick, one-meter long plastic collimators, with 10 cm × 10 cm internal aperture, were used along the line of sight of the scintillators in order to reduce stray neutrons hitting the detector.

The three plastic scintillators discussed in this paper are EJ232Q (0.5% quenching, 150 mm diameter, and 25 mm thick), BC422Q (1% quenching, 180 mm diameter, and 10 mm thick), and EJ410 (90 mm diameter and 16 mm thick). Calibration of EJ232Q and EJ410 scintillators was obtained in the same set of shots while using the two detectors inside the detector housing as shown in Fig. 1. However, the BC422Q scintillator was calibrated with the BDS in a separate set of shots where the BC422Q detector was kept alone inside the detector housing. Where EJ232Q and BC422Q have fast rise and decay times (sub-ns and a few ns, respectively), EJ410 has significantly slower decay of 200 ns. Each scintillator was coupled with a fast PMT for converting the light output from the scintillator into electrical signal as well as to amplify the signal for detection. The gain of PMT was controlled by varying the biasing voltage from shot to shot in order to avoid signal saturation. The detectors were attached to fast (6 GHz) oscilloscopes for recording the output from the PMTs.

III. BASICS OF SCINTILLATION PROCESS

Since neutrons do not interact via the Coulomb force, they do not produce excitations in the scintillation material directly. Therefore, neutron detection by scintillators relies on

![FIG. 1. Schematic of the experimental setup. The neutron beam generated from laser irradiated target or catcher propagates through various diagnostics before reaching the scintillator detectors (figure not drawn to scale). The scintillator detectors were placed face-on to each other inside a lead and plastic housing. The collimators, made of 5 cm thick plastic, were used before the detectors in order to reduce stray neutrons hitting the detector.](image-url)
secondary ionising particle generation in the scintillator material via elastic, inelastic, or nuclear reactions. Fast neutrons (generally ≥0.5 MeV) can efficiently produce recoil protons via elastic scattering in hydrogen-rich materials, whereas slow neutrons rely on nuclear reactions such as the (n,γ) or (n,α) reactions to produce ionisation. Materials such as LiI(Eu) or glass silicates doped with 9Li and 10B are therefore particularly well-suited for the detection of slow (thermal) neutrons.

The response of plastic scintillators to fast neutrons depends on several factors. The dominant mechanism for production of secondary ionisation radiation for incident neutron energy up to a few 10s of MeV is the proton recoil based on elastic n-p scattering. As neutrons of a given energy pass through the scintillator, they produce a spectrum of recoiled protons by direct elastic scattering with hydrogen atoms present inside the material. Energy of the recoiled protons varies from zero to a maximum value, which is equal to the incident neutron energy. Each recoiled proton then produces scintillation light, \( L_p(E) \), while travelling in the scintillator over a finite length depending on its kinetic energy. A part of the energy deposited by the protons is used for the scintillation process and the \( L_p(E) \) can be calculated with semi-empirical formulae described by Chou and Wright. Where Chou’s formula, given as

\[
\frac{dL_p(E)}{dx} = S \frac{dE}{dx} \left[ 1 + KB \frac{dE}{dx} + C \left( \frac{dE}{dx} \right)^2 \right]^{-1},
\]

which is a generalisation of the Birk’s formula, Wright formula,

\[
L_p(E) = A \int_0^R \ln \left[ 1 + \alpha \left( \frac{dE}{dx} \right) \right] dx,
\]

on the other hand uses a simpler model while attributing for quenching effects in scintillators. In the above formulae, \( S, KB, C, A, \) and \( \alpha \) are the fitting parameters to the experimental data, \( R \) is range of the proton inside the scintillator, and \( dE/dx \) is the proton energy loss in the scintillator material per unit length.

**IV. RELATIVE RESPONSE OF EJ232Q TO FAST NEUTRONS**

Although the semi-empirical formulae of Chou and Wright provide fairly accurate estimation of the light output, the statistical nature of particle scattering and energy loss mechanisms demands a thorough Monte Carlo approach in order to characterise the response of plastic scintillators to fast neutrons. Most of the commercially available plastic scintillators are experimentally calibrated for different types of ionising radiation, such as electrons, protons, and alpha particles, as provided by the manufacturers. However, the calibration of plastic scintillators for fast neutrons is often unknown, as is the case for EJ232Q and BC422Q used in our experiment. Therefore, we used a simple model based on a series of systematic Monte Carlo simulations, as described below, in order to obtain a relative response in terms of the light output by the whole spectrum of recoiled protons.

At first, the energy spectrum of recoiled protons produced by a given energy of neutrons in EJ232Q plastic (which is the same material used in BC422Q) was obtained by employing FLUKA simulations. The stoichiometric information about the EJ232Q plastic was taken from the manufacturer documentation. A series of FLUKA simulations were carried out by irradiating a small cube of the scintillator material (10 μm × 10 μm × 10 μm) with different neutron energies. For each neutron energy, the recoiled protons generated inside the cube were tallied. As an example, Fig. 2(a) shows the recoiled proton spectra produced by neutrons of three different energies.

**FIG. 2.** (a) Spectra of recoiled protons generated inside a 10 μm × 10 μm × 10 μm cube of EJ232Q scintillator for different incident neutron energies. (b) shows stopping range of protons inside EJ232Q scintillation material obtained by SRIM simulation. (c) shows scintillation light output for different proton energies calculated for EJ232Q scintillator by using the formulas of Chou and Wright, as well as for a few different scintillators obtained from the literature. (d) shows the relative weighted average light output from the EJ232Q obtained by using Eq. (3), where the recoiled proton spectrum is shown in (a) and \( L_p(E) \) is shown in (c).
The next step of calculation involves finding the stopping range of protons in the EJ232Q material, which was simulated using Stopping and Range of Ions in Matter (SRIM) Monte Carlo simulation. The stopping range was calculated by obtaining \(\frac{dE}{dx}\) as a function of incident energy of protons while passing through a very thin (1 \(\mu\)m) layer of EJ232Q, as shown in Fig. 2(b). Using the stopping range of protons, \(L_p(E)\) for the EJ232Q scintillator material was calculated for different proton energies by using the semi-empirical formulae of Chou and Wright, as shown in Fig. 2(c). As one can see, the \(L_p(E)\) from both formulae closely agree with each other and with the data found in literature for similar plastic scintillators. It also closely matches with the data provided by the manufacturers.

Finally, \(L_n(E_n)\), the weighted average scintillation light output for a given neutron energy \(E_n\) is obtained by integrating the light output from all the recoiled protons generated by the fast neutrons inside the scintillator. Mathematically, one can write

\[
L_n(E_n) = \int_{0}^{E_{\text{max}}=E_n} \frac{dN_p}{dE} \mid_{E_n} L_p(E) \, dE, \quad (3)
\]

where \(\frac{dN_p}{dE}\) is the recoil proton spectrum shown in Fig. 2(a) and \(L_p(E)\) is the light output by different energy protons in the scintillator, shown in Fig. 2(c). By following the steps mentioned above, the weighted average light output from the scintillator was obtained for different incident neutron energies as shown in Fig. 2(d).

Since the BC422Q is the same scintillator material as EJ232Q, produced by a different company, we used the same detector response for EJ232Q and BC422Q in the data analysis described below. However, for the EJ410 scintillator, we used the relative response of the scintillator to different neutron energies provided by the manufacturer.

V. CALIBRATION OF THE SCINTILLATOR DETECTORS

The TOF arrangement allows us to distinguish the signal produced by the neutrons from the gamma-rays, produced during the laser interaction, by their difference in arrival time at the detector. The prompt gamma flash in the raw data at early time, as can be seen in Fig. 3(a), serves as a precise temporal fiducial to identify the time of neutron generation at the source. Using this temporal reference, the time axis of the raw data was converted into neutron energy by taking into account the distance between the source and the detector. Due to the large distance (\(\sim 11\) m) between the source and the detector employed in the experiment, the signals due to gamma radiations and neutrons were well separated in time, as can be seen in Fig. 3(a), which allowed us to discard unambiguously the gamma signal from the raw data before processing further.

The scintillator detectors used in the experiment were coupled with fast photo-multiplier tubes in order to convert the optical photons from the scintillator into electrical signal with a gain commensurate to the applied bias voltage. The gain characteristics of the PMTs are calibrated precisely by the manufacturers, which were used in the data analysis, following the background subtraction of the raw data and deconvolution of the signal by scintillation temporal profile. Assuming the light collection by detector geometry, transmission through light guide and quantum efficiency of the PMT are constants for a given detector, each point in the processed data now corresponds to the light emitted by the scintillator due to the neutrons within the respective energy bin. Since the energy bins are typically very small (for example, \(\sim 1\) keV at 1 MeV and \(\sim 10\) keV at 10 MeV), the relative number of neutrons in a given energy bin, \(dN(E_n)\), is now obtained by dividing each point of the data by the corresponding \(L_n(E_n)\) (given in Fig. 2(d)). Finally, the neutron spectrum (\(dN/dE\)) was obtained by dividing each data point by the width of the respective energy bin. Fig. 3(b) shows neutron spectrum obtained from the data collected by the EJ232Q and EJ410 detectors. However, the spectrum does not yield the absolute number of neutrons hitting the detector because of the various unknown factors, such as light collection by the detector housing, attenuation of scintillation photon by the light guide, and quantum efficiency of the PMT photocathode. Therefore, it is important to cross-calibrate the detectors against a standard dosimeter. In our case, absolute flux calibration was carried out against the BDS fielded in the same shot along the same direction of observation.

FIG. 3. (a) Typical raw data obtained by the two different scintillators, EJ232Q and EJ410, in the same shot. The sharper peak in the signal near to zero time corresponds to prompt gamma irradiation, while the following peak corresponds to neutron signal. (b) shows deconvolved neutron spectra from the raw signal shown in (a), after accounting for PMT gain and scintillator response.

FIG. 4. Transmission in % of different energy neutrons from source to the detector obtained by FLUKA simulations for the setup shown in Fig. 1.
Bubble detectors (BD) are passive detection devices that provide neutron response proportional to the number of bubbles formed in the detector. The response of the bubble detectors depends on the kinetic energy of the incident neutrons and the composition of the BD medium. Therefore, the BDS consists of six sets of bubble detectors calibrated for different energy ranges of neutrons. After exposure to the broadband neutron source, the number of bubbles in each of the bubble detector was manually counted and the neutron spectrum is obtained by deconvolving the number of bubbles in each set of detectors against their calibration curves provided by the manufacturer.

In order to cross-calibrate the scintillator detectors against the data obtained in the BDS, it is important to assess the transmission and scattering of neutrons by the various objects (including the BDS) they encounter along the scintillators’ line of sight. Since the scintillators are placed at a significantly large distance (~11 m) from the BDS, any small angle scattering of neutrons by the intermediate objects would prohibit the scattered neutrons from reaching the scintillator detector. On this basis, the neutron propagation through the system was simulated using FLUKA in order to estimate transmission to the detector as a function of neutron energy, as shown in Fig. 4.

In the simulation, the neutron source was defined as a pencil beam with no divergence. The simulation was set by considering all different objects on the line of sight to the scintillator detector, including the chamber wall and the detector housing. A series of simulations were carried out for different neutron energies at the source and the neutron spectra were tallied. Due to the heavy shielding (typically, 5 cm of lead and 5 cm of plastic) used around the detectors (to suppress the noise level in the detector by gamma rays and thermal neutrons), the scattered neutrons in the vicinity of the detectors can in principle contribute towards the signal for that energy in a TOF mode. In order to account for the scattered neutrons, two sets of simulations were done for each energy, one with and one without the lead and plastic shielding around the detector. The neutron spectra at the detector obtained from these two simulations were subtracted in order to estimate the spectra of scattered neutrons reaching the detector due to the detector housing. The transmission factor is then calculated by taking into account the scattered neutron spectra and the number of neutrons of the input energy, convolved with the detector detection response of the detector.

The neutron transmission calculated above was used in the TOF data analysis in order to obtain the neutron spectrum at the BDS plane from the neutron spectrum at the scintillator plane, as shown in Fig. 3(b). In this way, a direct comparison between neutron flux obtained by the scintillator and BDS can be acquired for an absolute calibration. A typical data set is shown in Fig. 5(a) where the neutron spectra recorded from EJ232Q and EJ410 scintillators are shown in comparison with the spectrum obtained by BDS, after applying a constant factor in order to match the neutron flux with that measured by the BDS. One can see a good agreement between the spectra in terms of spectral shape, which remains fairly consistent.

The comparison between the neutron flux obtained from different scintillators with the one measured absolutely by the BDS over a number of shots is shown in Figure 5(b). In this case, we have chosen the high energy range (2.5–20 MeV) for flux comparison in order to avoid discrepancies due to the down-scattered (lower energy) neutrons detected by the BDS, which is a time integrated spectrometer. As can be seen from Fig. 5(b), the data points follow a linear trend over a range of neutron fluxes, the slope of which can be used as the final calibration factor for the scintillators after applying other corrections, such as the scintillator detection response, the PMT gain, and the transmission function for the setup.

VI. CONCLUSION

Employing a sub-ns neutron source driven by high power laser and absolutely calibrated gamma insensitive bubble detector spectrometers, we have carried out a systematic calibration of three types of plastic scintillators, namely, EJ232Q, BC422Q, and EJ410. Cross-calibration of the scintillators was done by implementing the neutron detection response (modeled semi-empirically for EJ232Q and BC422Q and using the one obtained from the manufacture for EJ410), PMT gain factors obtained from the manufacturers, and transmission of neutron through our experimental setup as simulated by FLUKA. Over a number of shots, spanning over a wide range of neutron fluxes between $10^8$ and $10^{10}$ n/sr for 2.5–20 MeV neutrons, the comparison between the results obtained from the TOF and the BDS data follows a linear fit, which provides the final calibration factor for the scintillators.
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